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The Effect of Reducing Irradiation Temperature on the Structure and Radiation Embrittlement Mechanism of RPV Steels

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Abstract

A complex research of RPV 15Kh2NMFA class 1 steel in different states was performed: in the initial state and after irradiation at different temperatures (50-300°C). Low temperature (50-140°C) irradiation was shown to cause the formation of dislocation loops of ultrahigh density. At the same time radiation-induced precipitates, being the main radiation-induced hardening phase at irradiation temperature of VVER reactors (~300°C), were absent. This leads to a higher radiation embrittlement rate after low-temperature irradiation as compared with irradiation at operating temperatures of VVER-1000. The radiation embrittlement coefficient values are 8.7 and 1.45 for (120-140)°C and (290-315)°C irradiation temperature, respectively.

1. Introduction

Numerous studies demonstrated occurrence of two embrittlement mechanisms, which relative contribution can change during irradiation: hardening mechanism - (formation of radiation defects and radiation-induced precipitates) and non-hardening mechanism - (formation of grain boundary impurities segregation).

Irradiation temperature has a significant influence on the processes in steel that control each of the embrittlement mechanisms. On the one hand, the temperature decrease has to slowdown the point defect of crystal lattice and decrease the probability of vacancy – interstitial atom annihilation that will increase the radiation damage effectiveness, i.e. increase the rate of radiation defects accumulation. On the other hand, the slowdown of the point defect and atoms migration, i.e. the diffusion processes slowdown, can results in decreasing both the intensity of radiationinduced phase transformation and the rate of grain boundary segregation accumulation and, consequently, has to affect the radiation embrittlement rate of the steel

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at low-temperature irradiation as compared to similar steels that were irradiated at higher temperatures.

The most important characteristics for RPV-steels of nuclear VVER-type reactors are their resistance to thermal and radiation embrittlement. Radiation component of the ductile to brittle transition temperature (DBTT) shift can be represented as:

$$\Delta T_K^F = A_F \cdot \left(\frac{F}{F_0}\right)^m \tag{1}$$

where ΔT_K^F – radiation component of DBTT shift, F – fluence, F_0 – normalization fluence, A_F – radiation embrittlement coefficient, m – experimentally determined parameter.

The influence of irradiation on the steel structure degradation and the coefficient of radiation embrittlement A_F within the temperature range (50-315)°C was investigated.

2. Materials and methods

The paper focuses on VVER-1000 RPV 15Kh2NMFA class 1 base and weld metal specimens in the initial state and after accelerated irradiation in a research reactor up to fast neutron fluences (E>0.5 MeV): at irradiation temperatures (50-140)°C up to fluences(4,5-8) $10^{22}m^{-2}$; at irradiation temperatures (290-310)°C up to fluences ~4,510²³m⁻². Table 1 shows the chemical composition of 15Kh2NMFA class 1 steel.

TABLE 1: Chemical composition of 15Kh2NMFAA class 1 steel base metal (BM) and weld metal (WM).

Material	Composition, wt. %									
	С	Mn	Si	Ni	Cr	Мо	V	Cu	Р	S
BM	0.16	0.53	0.31	1.23	2.32	0.62	0.11	0.03	<0.005	<0.003
WM	0.08	0.98	0.24	1.18	1.92	0.59	0.017	0.04	<0.008	0.006

The mechanical tests (static tension and impact tests) were carried out to determine the yield strength change and DBTT temperature shift (ΔT_K). Furthermore, to determine radiation-induced structural components - radiation-induced precipitates and radiation defects - analytical research methods were used: transmission electron microscopy (TEM) («Titan 80-300», FEI, USA); scanning electron microscopy (SEM) («Supra 40-VP», «Merlin», Zeiss, FRG) and atom-probe tomography (APT) («LEAP 4000 HR» Cameca, France). Grain boundary segregation was studied by Auger electron spectroscopy (AES) (PHI 700, Physical Electronics, Japan-USA).



3. Phase composition study of 15Kh2NMFA class 1 steel and its weld in the initial state and after irradiation in (50-315) °C temperature range

To reveal regularities of radiation hardening the structural studies were performed for materials in the initial state and after irradiation to a variety of fast neutron fluences in different temperature ranges.

3.1. The study of initial hardening carbide and carbonitride phases

Phase analysis of steel specimens in the initial state showed (according to micro diffraction patterns) that the base metal along with tempered bainite contains hardening phases: Me_7C_3 , $(Mo,Cr)_2C$ type carbides, V(C,N) carbonitrides and non-metallic inclusions (manganese sulfides MnS and silicon oxides SiO₂). The weld metal contains no carbonitrides. The number density of carbide phases also differs for the weld and base metal (see Table 2) due to the higher carbon content in the base metal.

Material	State	Number density (ρ) and average size (d) of second phases						
		Me_7C_3 ,		V(C,N)		Me ₂ C		
		d, nm	N, 10 ¹⁹ m ⁻³	d, nm	N, 10 ²² m ⁻³	d, nm	N, 10 ²⁰ m ⁻³	
BM	Initial	40-50	4.7±0.6	6-10	1.3±0.2	15-25	1.0 <u>+</u> 0.2	
BM	$F_1 = 5.12 \cdot 10^{22} \text{m}^{-2}$	45-55	5.2±0.7	4-8	1.4±0.3	15-25	0.8±0.2	
BM	$F_2 = 8.5 \cdot 10^{22} m^{-2}$	45-55	5.5±0.7	4-8	1.1±0.3	20-30	1.2 <u>+</u> 0.2	
WM	Initial	45-55	1.9±0.2	-	-	31±4	4.0±1.1	
WM	$F = 4.0 \cdot 10^{21} \text{m}^{-2}$	50-60	2.1±0.3	-	-	25-35	3.0±1.1	

TABLE 2: The number densities N and average sizes d of initial hardening phases in the initial state and after irradiation for base and weld metal specimens in (50-315)°C temperature range.

Thus, the irradiation did not cause a significant change of the initial hardening phases in the studied base and weld metal specimens: the number density and size of second phases match the corresponding values in the initial state (within the experimental data scatter). This means that the carbide and carbonitride phases do not additionally contribute to radiation hardening (the yield strength change) after irradiation in the specified temperature range.



3.2. The study of radiation-induced hardening phases and grain boundary segregation after irradiation in (50-140) °C temperature range

In 50-140°C temperature range radiation-induced structural changes are mostly caused by formation of different types of dislocation loops with no precipitates formation. Figure 1 and Table 3 show the absence of radiation-induced hardening phases after low temperature irradiation in contrast to the irradiation in (290-315)°C temperature range.



Figure 1: Radiation-induced structural elements at different irradiation temperature.

Table 3 and fig. 1 show that low-temperature irradiation (50-140) °C leads to significantly higher number density of dislocation loops as compared to the one for VVER-type temperature irradiation (\sim 300 °C). It can be seen that low-temperature irradiation up to \sim 5-8 times lower fast neutron fluence leads to formation of \sim 250-500 times higher density of dislocation loops with somewhat smaller sizes.

This leads to a greater rate of radiation hardening than for VVER-1000 steel irradiated at ~300 °C: a close yield stress shift is achieved after irradiation at ~ 300 °C at a fluence



Irradiation temperature, °C	Fluence, 10 ²² m ⁻²	Dislocation loops		Precipitates		
		Number density, 10 ²¹ m ⁻³	<d>, nm</d>	Number density, 10 ²³ m ⁻³	<d>, nm</d>	
(50-70) °C	6	1100±200	3±1	-	-	
(120-140) °C	8.5	600 <u>+</u> 100	4±1	-	-	
(290-315)°C	45.3	2.5±0.8	5±2	1.4 <u>±</u> 0.2	2.5±0.5	

TABLE 3: Number densities N and average sizes <d> of radiation-induced structural elements in 15Kh2NMFA class 1 steel after irradiation at different temperatures.

of an order of magnitude greater (see fig. 2). Thus, lowering the irradiation temperature led to a significant increase of radiation hardening.



Figure 2: The dose dependence of yield strength for various irradiation temperatures.

The average loops size increases with the irradiation temperature increase. Thus, radiation hardening is solely due to dislocation loops accumulation. At this, its density is the higher, the lower is irradiation temperature.

TEM-studies of steels after irradiation at different temperatures revealed mainly the <100> Burgers vector of dislocation loops for all the studied steels but dislocation with <111> Burgers vector were also observed.

Dislocation loops in all the studied steels are mainly interstitial (its density is 1.5 times higher than for vacancy-type dislocation loops).

It was shown in [1] that the hardening due to <100> and 1/2<111> types interstitials dislocation leads to a strong hardening in BCC metals.



Vacancy loops are also known [2] to cause higher radiation hardening as compared to clusters. The higher is irradiation temperature, the lower is point defects contribution to radiation hardening.

That is why hardening is controlled by dislocation loops at low temperatures and their contribution is higher than at higher temperatures.

Fractographic studies demonstrated no brittle intergranular fracture in specimens irradiated at low temperature, indicating absence of grain boundary segregation. AES study neither revealed any grain boundary segregations as far as long-term liquid nitrogen cooling of the specimens has not resulted in getting a brittle intergranular fracture, and therefore gave no possibility to measure the grain boundary segregation. Thus, the results demonstrate that low-temperature irradiation causes no detectable grain boundary segregation due to low diffusion mobility of atoms (primarily phosphorus).

In sum, irradiation temperature effect on nanostructure evolution is the following.

3.3. Low-temperature irradiation (50-140°C)

In this temperature range radiation-induced structural changes are mostly formation of different types of dislocation loops with no precipitates formation. The average loops size increases with the irradiation temperature increase. Thus, radiation hardening is solely due to dislocation loops accumulation. At this, its density is the higher, the lower is irradiation temperature.

There is no segregation due to low diffusion mobility of atoms (primarily phosphorus). In this regard only hardening mechanism is responsible for the ductile-to-brittle transition temperature shift.

3.4. VVER-type RPV irradiation temperature (290 - 315°C)

In this temperature range radiation-induced structural changes are both formation of dislocation loops and precipitates. Besides, there is formation of grain boundary segregation. Radiation embrittlement is due to both hardening and non-hardening mechanisms.

Figure 3 shows the impact test results for determining DBTT shift for 15Kh2NMFA class 1 steel specimens irradiated at different temperatures.

Thus, the observed structural changes under the influence of low-temperature irradiation lead to a significantly greater extent of radiation embrittlement compared with





Figure 3: The dose dependence of the DBTT shift ΔT for 15Kh2NMFA class 1 steel at different irradiation temperature.

irradiation at ~300 °C, the operating temperature of VVER-1000 type RPV. Indeed, radiation embrittlement coefficient A_F is 8.7 and 1.45 for irradiation at temperatures (120-140)°C and (290-315)°C, correspondingly, which significantly exceeds the normalized A_F values for these steels at the irradiation temperature (290-315)°C [24].

4. Conclusions

- 1. 15Kh2NMFAA steel specimens were studied in initial state and after accelerated fast neutron (E> 0.5 MeV) irradiation in IR-8 up to fluences ($6 \cdot 10^{22} \text{m}^{-2} \div 4.5 \cdot 10^{23} \text{m}^{-2}$) in the temperature ranges (50-140)°C and (290-315)°C.
- 2. Radiation hardening of 15Kh2NMFAA RPV steel at (290-315)°C temperature irradiation and fluences up to $\sim 4 \cdot 10^{23} \text{ m}^{-2}$ is mainly due to radiation-induced precipitates formation of high density and some contribution of radiation defects, while radiation hardening at low-temperature irradiation (50-140)°C is caused only by radiation defects formation – dislocation loops of ultrahigh density.
- 3. This leads to a higher radiation embrittlement rate at low-temperature irradiation: A_F values are 8.7 and 1.45 for irradiation temperature (120-140)°C and (290-315)°C, respectively.



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